

Self-Organization in Burning Plasmas

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The ITER experiment, currently under construction, will allow us to study burning plasmas



- Overall programmatic objective:
 - to demonstrate scientific and technological feasibility of fusion energy for peaceful purposes
- Principal goal:
 - to design, construct and operate a tokamak experiment at a scale which satisfies this objective
- ITER is designed to confine a DT plasma in which α-particle heating dominates all other forms of plasma heating
 - \Rightarrow a burning plasma experiment





Fusion energy research focusses on the development of DT burning plasmas



+ 20% of Energy (3.5 MeV)

+ 80% of Energy (14.1 MeV)

$1 \text{ keV} = 1.16 \times 10^7 \text{ K}$



 $^{2}D + ^{3}T \Rightarrow ^{4}He (3.5 \text{ MeV}) + ^{1}n (14.1 \text{ MeV})$



The Tokamak is the most advanced toroidal magnetic confinement configuration



- Inject gas into high vacuum chamber with a strong toroidal magnetic field (e.g. B_T ~ 5 T)
- Induce a toroidal electric field through transformer action, avalanche ionization produces plasma current creating poloidal magnetic field

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Combination of toroidal and poloidal fields produces helical field for plasma confinement:

 $\mathbf{F} = q(\mathbf{E} + \mathbf{v} \times \mathbf{B})$

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The conditions required for significant fusion power gain are well understood

Temperature - T _i :	1-2 × 10 ⁸ K (10-20 keV) (~10 × temperature of sun's core)
Density - n _i :	1 × 10 ²⁰ m ⁻³ (~10 ⁻⁶ of atmospheric particle density)
Energy confinement time - τ_{E} : $\tau_{E} = \frac{W_{th}}{P_{loss}} = 3 \frac{\int nkT dV}{P_{loss}}$	<i>few seconds</i> (plasma pulse duration ~1000s)
Fusion power amplification:	$Q = \frac{Fusion Power}{r} \sim n_i T_i \tau_F$

Fusion power amplification: $Q = \frac{Pusion Power}{Input Power} \sim n_i T_i \tau_E$ \Rightarrow Present devices: $Q \leq 1$ \Rightarrow ITER: $Q \geq 10$ \Rightarrow 'Controlled ignition': $Q \geq 30$

DT experiments on existing major facilities have laid the basis for studies of burning plasmas

- Existing experiments have achieved nTτ values
 - ~ 1×10²¹ m⁻³skeV
 - ~ Q_{DT} = 1
- JET and TFTR produced DT fusion powers of >10 MW for ~1 s
- ITER is designed to a scale which should yield Q_{DT} > 10 at a fusion power of ~ 500 MW for ~ 400s, allowing exploration of the burning plasma regime under stationary conditions



Heat and particle transport in fusion plasmas is generally dominated by turbulence

- A well developed 'neoclassical' theory of transport in toroidal plasmas has been derived from analysis of collisional processes:
 - unfortunately, it doesn't do a good job of describing heat and particle transport across the magnetic field
 - \Rightarrow turbulence normally dominates
- Free energy available within the plasma can generate turbulence and magnetohydrodynamic instabilities (mhd) which reduce plasma confinement quality
 - small scale turbulence dominates collisional transport processes
 - physics-based quantitative predictions of transport processes (and global confinement) not yet possible
- Parallel transport (along magnetic field lines) equilibrates heat and particle fluctuations rapidly, but measured values of perpendicular heat transport typically exceed 'neoclassical' predictions by:
 - about an order of magnitude for the ion channel
 - at least two orders of magnitude for the electron channel

Empirical predictions of τ_E are derived from 'scaling' analysis

- Since turbulent transport is difficult to predict quantitatively:
 - we use scaling experiments to predict the level of energy confinement in future experiments such as ITER



A bifurcation in the behavior of $\tau_{\rm E}$ is observed under certain conditions

- It is found that the plasma confinement state (τ_E) can bifurcate:
 - two distinct plasma regimes, a low confinement (L-mode) and a high confinement (H-mode), result



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 - this phenomenon has been shown to arise from changes in the plasma flow in a narrow edge region, or pedestal, just a few centimetres wide



Evidence for self-organization in observed in 'profile stiffness'

- Experimental evidence over several decades suggested that plasma profiles in tokamaks increased in 'self-similar' fashion, e.g. electron temperature
- 'Profile stiffness' indicates that there is little change in temperature gradient in response to large changes in heat flux

5.0 = 5.0 5.0 = 1.0 0.5 = 0.5 0.5 = 0.8 MW ECH 0.1 = 0.2 = 0.4 = 0.6 = 0.8 = 1

F Ryter et al, Plasma Phys Control Fusion **43** A323 (2001)

ASDEX Upgrade

'Profile stiffness' reflects threshold behaviour expected for certain turbulent instabilities

- Studies show that experimental heat flux, q_i, rises sharply above a threshold value, or 'critical gradient', R.∇T_i/T_i = R/L_{Ti}
- This behaviour corresponds to that predicted for certain classes of turbulent instabilities
- Degree of 'stiffness' found to depend on plasma rotation implying influence of rotation on instability amplitude



P Mantica et al, Phys Rev Lett **102** 175002 (2009)

Turbulence saturation mechanism also shows evidence for 'self-organization'

- Massive computational simulations of microturbulence have confirmed predictions of analytic theory:
 - Fluctuating E × B driven 'zonal flows' develop non-linearly within the turbulence and produce 'shearing' of the turbulent eddies, reducing plasma transport
 - Zonal flows act as saturation/ selfregulation mechanism of the turbulence via 'shearing' of turbulent eddies
- Existence of zonal flows confirmed in experimental measurements

Z Lin et al, Science 281 1835 (1998)



Simulations of poloidal distribution of fluctuation potential, $e\Phi/T_i$:

- (A) E × B flows included
- (B) E × B flows suppressed

The H-mode bifurcation is understood in terms of E × B turbulence suppression

- The H-mode is correlated with a change in rotational shear at the plasma edge associated with the development of a radial electric field:
 - turbulence is stabilized when the shearing rate for the modes is of order of the mode growth rate:

 $\omega_{E \times B} \sim \gamma_{max}$

 However, debate is ongoing as to whether the shearing process is dominated by zonal flows (resulting from 'Reynolds stress') or velocity shear driven by 'neoclassical' E-field generation (ion orbit loss or pressure gradient)



Summary

- Extensive physics studies of magnetically confined toroidal plasmas have laid the basis for the production and study of burning plasmas in ITER
- Experimental, theoretical and computational analysis of heat and particle transport are unravelling the complexities of turbulence driven transport:
 - self-organization phenomena are critical to an understanding of the nonlinear behaviour of turbulence and its influence on plasma confinement quality

• Self-organization processes are likely to assume even greater importance in the burning plasma regime:

- internal heating by α -particles and the interaction with α -driven mhd modes may introduce additional non-linear aspects of plasma behaviour
- possible modes of plasma operation allowing true steady-state operation of tokamak plasmas may also give rise to self-organization phenomena